Construction and Implementation of an Energy-Dependent Instrument Response Function for Accurate Analysis of Neutron Time-of-Flight Data



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It is necessary to use an energy-dependent IRF to accurately infer parameters from nTOF data that span a wide range of energies

- The shape of the IRF changes with incident neutron energy
- The matrix representation of the convolution is used to incorporate the energydependence of the IRF
- The energy-dependent IRF is needed to infer the correct lifetime and branching ratios of nuclear states (e.g., ⁵He states inferred from TT data*)

Accurate neutron spectroscopy is now possible over a wide range of energies (e.g., areal density in cryogenic experiments, inelastic reactions involving DT neutrons on D or ⁷Li).

IRF: instrument response function nTOF: neutron time of flight * M. Gatu Johnson *et al.*, Phys. Rev. Lett. 121 042501 (2018).



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Collaborators



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The measured signal from an nTOF detector can be written as a function of time of arrival at the detector





Previous work assumes that the IRF varies slowly with incident neutron energy

- If the IRF varies slowly with neutron energy*,**, $R(E, t t') \approx R(E = E_0, t t') = R_0(t t')$, so the fit function becomes $\mathbf{m}(t) = C \int_0^{t'} \frac{dN}{dE} [E(t')] \frac{dE}{dt'} [E(t')] R_0(t - t') dt'$
- The integral can be written as a matrix multiplication

$$m(t_k) = \sum_{i=1}^k P[E(t'_i)]R_0(t_k - t'_i) \to \overrightarrow{m} = \overrightarrow{T P}$$

where T is a Toeplitz matrix of the response vector with shape $N_m \times N_p$

and $C \frac{dN}{dE} [E(t')] \frac{dE}{dt'} [E(t')] \Delta t'_i = P[E(t'_i)]T$



** E. P. Hartouni et al, Review of Scientific Instruments 87 11D841 (2016).



^{*} R. Hatarik et al., Journal of Applied Physics 118 184502 (2015).

The energy-dependent convolution replaces the matrix elements with rows that are energy dependent*

$$\mathbf{m}(t) = C \int_0^{t'} \frac{dN}{dE} [E(t')] \frac{dE}{dt'} [E(t')] R[E(t'_i), t_k - t'_i] dt' \to \mathbf{m}(t_k) = \sum_{i=1}^k P[E(t'_i)] R[E(t'_i), t_k - t'_i] \to \overline{\mathbf{m}} = \overline{T P}$$

Each column in the matrix represents a monoenergetic IRF

$$\mathbf{T} = \begin{bmatrix} r_{0,0} & 0 & \dots & 0 & 0 \\ r_{0,1} & r_{1,0} & \dots & \dots & \dots \\ r_{0,2} & r_{1,1} & \dots & 0 & 0 \\ \dots & r_{1,2} & \dots & r_{N_{p-1},0} & 0 \\ r_{0,N_r-1} & \dots & \dots & r_{N_{p-1},1} & r_{N_p,0} \\ r_{0,N_r} & r_{1,N_r-1} & \dots & \dots & r_{N_p,1} \\ 0 & r_{1,N_r} & \dots & r_{N_{p-1},N_{r-2}} & \dots \\ 0 & 0 & \dots & r_{N_{p-1},N_{r-1}} & r_{N_p,N_{r-2}} \\ \dots & \dots & \dots & r_{N_{p-1},N_r} & r_{N_p,N_{r-1}} \\ 0 & 0 & 0 & \dots & r_{N_p,N_r} \end{bmatrix}$$

* Z. L. Mohamed, O. M. Mannion, E. P. Hartouni, J. P. Knauer, and C. J. Forrest, submitted to Journal of Applied Physics



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A monoenergetic IRF can be constructed by convolving a measured x-ray response with a calculated neutron interaction response

- The x-ray response \rightarrow impulse response
- Neutron interaction response \rightarrow neutron transport through the detector
 - IRF shape is a function of neutron energy



Use of the energy-dependent IRF accurately infers the widths and masses of the ⁵He states from TT nTOF data*





^{**} M. Gatu Johnson *et al.*, Phys. Rev. Lett. 121 042501 (2018). ** B. Lacina, J. Ingley, and D. W. Dorn, Lawrence Livermore National Laboratory, Report UCRL-7769 (1965).

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Backup





Interpolation over the total IRF is more accurate than attempting to directly interpolate over the neutron interaction responses



- Generate a representative set of neutron interaction responses, then interpolate over the total IRF
- The x-ray response and neutron interaction responses must be start at t=0 and be area normalized to conserve correct timing and yields in the forward fit
- Uncertainty in the total IRF arises mainly from noise on x-ray IRF*
 - This is an uncertainty in the model, not the data (address by Monte Carlo)





The measured signal from an nTOF detector can be written as a function of time of arrival at the detector

• $\mathbf{m}(t) = C \int_0^t \frac{dN}{dE} [E(t)] \frac{dE}{dt} [E(t)] R(E, t) dt$

m(t) = the measured nTOF signal

t = time scale recorded by oscilloscope

C = calibration constant

dN/dE = neutron energy spectrum

dE/dt = Jacobian

R(E,t) = total IRF

• This can be rewritten as a function of the neutron's time of arrival at the detector (t') $m(t) = C \int_{0}^{t'} \frac{dN}{dE} [E(t')] \frac{dE}{dt'} [E(t')] R[E(t'_{i}), t_{k} - t'_{i}] dt'$



Synthetic TT data (based on real OMEGA TT data) can be used as an example

Results:

- Inferred mass isn't affected by choice of IRF (expected because mass~mean neutron energy)
- Use of energy-dependent IRF infers correct width for both states to within 2%
- Use of 2.45-MeV IRF causes ~22% decrease in inferred ⁵He ground state width, ~10% increase in inferred first excited state width
 - Expected because ground state mass ~0.4 MeV and first excited state mass ~2.5 MeV
- Use of 14.03-MeV IRF inferred correct width for both states to within a few percent
 - Width of 14.03-MeV IRF was within 300 ps of width of 8.5-MeV IRF (ground state neutron energy)
 - This effect only occurs because of this specific detector configuration/material/distance and this specific nuclear data set
- More complex and more novel data with many resonances (e.g., n(D,p)2n or ⁷Li data) requires matrix IRF



Synthetic TT data (based on real OMEGA TT data) can be used as an example

- The properties (mass and width/lifetime) of the ⁵He ground state and first excited state can be inferred from a forward fit to TT nTOF data
 - Ground state at ~8.5 MeV neutron energy, 0.4 MeV width, excited state at ~6.5 MeV neutron energy, 2.5 MeV width
- Synthetic data was used to compare inferred values to input used to generate synthetic data
 - Monoenergetic IRF's at 2.45-MeV (DD) and 14.03-MeV (DT) can be compared to energy-dependent IRF





Synthetic TT data (based on real OMEGA TT data) can be used as an example

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Minimization in the forward fit must include both Gaussian- and Poissondistributed uncertainties



- Gaussian-distributed uncertainties from digitization noise and Poisson-distributed uncertainties from predicted number of neutrons per time bin
 - Poisson component can only be determined based on the current iteration of the forward fit
- Minimize $\chi^2 = \sum_{i=1}^{\chi} \frac{(fit_i data_i)^2}{\sigma_i^2}$ where $\sigma_i^2 = \sigma_{scope}^2 + \sigma_{Vi}^2$
 - First term is Gaussian-distributed digitization noise, second term includes Poissondistributed uncertainties based on each iteration of the forward fit
 - If fit function has a form resembling $V(t) \sim k \{ [s(E)a(E)\frac{dN}{dE_{4\pi}}\frac{dE}{dt}] \otimes IRF \}$, then the number of detected neutrons is $n(t) = a(E)\frac{dN}{dE_{4\pi}}\frac{dE}{dt}$

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$$V(t) \sim k \{ [s(E) \bullet a(E) \bullet n] \otimes IRF \} \rightarrow \sigma_{Vi}^2 = V_i^2 \{ \frac{\sigma_k^2}{k^2} + \frac{(n \bullet s^2 \bullet dt) \otimes IRF}{[(n \bullet s) \otimes IRF]^2} \}$$





The uncertainty introduced by the IRF must be determined by Monte Carlo



- Minimization in the forward fit must include both Gaussian- and Poisson-distributed uncertainties in the data
- The main source of uncertainty in the total IRF comes from digitization noise on the measured x-ray response
 - I.e., the measured x-ray response is one of an ensemble of responses we could have measured given some unknown "true" x-ray response
 - There are Poisson uncertainties in the calculated neutron interaction responses, but these can be minimized by running simulations with high statistics
- In relation to the forward fit, this is an uncertainty in the model, not an uncertainty in the data
- The most straightforward way to address this type of uncertainty is by Monte Carlo
 - Perturb the x-ray IRF, build a series of new matrix IRFs and histogram the results

